

Introduction to the Introduction to Nuclear Materials

22.14 – Intro to Nuclear Materials February 3, 2014

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Materials in Nuclear Systems

- Course Goals
- Intro to Nuclear Materials Degradation
- LWR Specifics
- Fast Reactor Specifics
- Fusion Reactor Specifics
- Waste Canister Specifics
- Material Degradation and Failure

Course Goals

 Obtain a basic knowledge of key degradation phenomena and material limitations in nuclear energy technologies

• Obtain a basic knowledge of mechanical properties; stress-strain relationship, plasticity, slip, fracture

Course Goals

• Understand and quantify radiation damage; energy transfer, displacement cross-sections, displacement rate

• Understand radiation effect mechanisms; radiation induced segregation, swelling, hardening, embrittlement

Know Your Systems – LWRs

Revisit: Mar. 17 (System Recap)



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PWR Material Map



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Know Your Systems – Fast Reactors

Revisit: Mar. 17 (System Recap)



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Know Your Systems – Tokamak (Fusion), ITER

Revisit: Mar. 17 (System Recap)



Schematic diagram of the ITER fusion reactor. © ITER Organization. All rights reserved. This content is excluded from our Creative Commons license. For more information, see http://ocw.mit.edu/help/faq-fair-use/.

http://www.odec.ca/projects/2007/ewar7j2/Nuclear%20Power%20%28Fusion%29.htm

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Know Your Systems – Tokamak (Fusion), MIT Alcator

Revisit: Mar. 17 (System Recap)



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Know Your Systems – Waste Canister

Revisit: Mar. 17 (System Recap)



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Overall Materials Science Philosophy



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Source: Wikimedia Commons, Materials Science Tetrahedron

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Background: Grains



Fe-12Cr-2Si ferritic alloy grains, 50x, etched with Kalling's reagent

Revisit: Feb. 10 (Crystal Structures)

- Most metals are granular in nature
- Grains are single crystals
- Grain boundaries separate them

Background: Phases



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Pearlite, or ferrite/cementite, phases in carbon steel

Image source: http://hsc.csu.edu.au/engineering_studies/ application/civil/1-1/answers.html **Revisit: Feb. 10 (Crystal Structures)**

- When elements mix, different **phases** form
- Phase: A coherent composition & microstructure
- Includes alloys, intermetallic
 compounds...
- Example: Steel (pearlite)

Background: Phase Diagrams

Revisit: Feb. 12 (Phase Diagrams)



application/civil/1-1/answers.html

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Background: Phase Transitions

Revisit: Feb. 12 (Phase Transitions)



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Iron⁻Carbon Binary Phase Diagram Image source: http://hsc.csu.edu.au/engineering_studies/ application/civil/1-1/answers.html

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Steel Heat Treatment and Microstructure

Revisit: Feb. 12 (Phase Transitions) Also see 22.71J (Physical Metallurgy)



Normalized Ductile fracture, tough, not very hard

Larger grains, no/few cracks or major defects



Quenched Very hard, NOT tough, brittle fracture

Small grains, can form microcracks



Tempered

Best of both, fairly hard, fairly tough, strong

Medium grains, partially relaxed internal stresses

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Heat Treatment

Revisit: Feb. 12 (Phase Transitions) Also see 22.71J (Physical Metallurgy)



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Japanese sword tip, with heat treatment line

Source: http://fudoshinken.com/?page_id=50

- Annealing: High heat to relax the microstructure
 - **Quenching**: Rapid cooldown to freeze microstructure
- **Tempering**: Low, slow heat for balance of properties
- Proper heat treatment is *essential* to correct material properties
- Incorrect treatment can be *disastrous*

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Predicting Change: TTT

Revisit: Feb. 12 (Phase Transitions) Also see 22.71J (Physical Metallurgy)



Courtesy of Ronald D. Kriz. Used with permission.

Source:

www.sv.vt.edu/classes/MSE2094_NoteBook/96ClassProj/examples/kimttt.html

- Time Temperature Transformation diagram
- Predict

 microstructure,
 properties vs. time at
 temperature
 (kinetics)
 - Choose your heat treatment & effects

Lecture 1, Page 18

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Welding and the HAZ



HAZ Cracking in a BWR

Fig. 1-1, 1-3 (a) and (b) in Ford, F. Peter. "Environmentally-Assisted Degradation of Structural Materials in Water Cooled Nuclear Reactors." *Advanced Nuclear Technology International Europe AB*, October 2006 removed due to copyright restrictions.

P. Ford. "Environmentally-Assisted Degradation of Structural Materials in Water Cooled Nuclear Reactors." ANT International, 2006.

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The Davis-Besse Incident

Fig. 1-1, 1-3 (a) and (b) in Ford, F. Peter. "Environmentally-Assisted Degradation of Structural Materials in Water Cooled Nuclear Reactors." *Advanced Nuclear Technology International Europe AB*, October 2006 removed due to copyright restrictions.

P. Ford. "Environmentally-Assisted Degradation of Structural Materials in Water Cooled Nuclear Reactors." ANT International, 2006.

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Young's Modulus (E)

Revisit: Feb. 26 (Mechanical Properties)

Measures elastic deformation vs. stress

Source: Wikimedia Commons

 $\sigma = F/A$ 0.2% Image courtesy of BenBritton on Wikimedia. License: CC-BY-SA. This content is excluded from our Creative Commons license. For more information, see http://ocw.mit.edu/help/faq-fair-use/.

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Grain Size vs. Strength

Revisit: Feb. 26 (Mechanical Properties)

- Microstructure changes many properties
- Smaller grain size leads to higher strength
- Called the Hall-Petch effect



Hall-Petch effect of grain size vs. strength





Fracture Toughness

Revisit: Feb. 26 (Mechanical Properties)



Source: Wikimedia Commons

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Fracture Toughness (K_{Ic})

Revisit: Feb. 26 (Mechanical Properties)

$$K_{Ic} = Y_1 \sigma^* \sqrt{\pi c} \text{ or } K_{Ic} = Y_2 \frac{F^*}{bw} \sqrt{\pi c}$$

Resistance to crack propagation

- -Y₁, Y₂ are geometric factors near 1
- $-\sigma^*$, F^{*} are critical stress and force, respectively

Source: inventor.grantadesign.com



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Toughness (G_c)

Revisit: Feb. 26 (Mechanical Properties)

Measures the energy it takes to separate a material Remote stress σ

$$K_{Ic} = \sqrt{EG_c}$$



Source: inventor.grantadesign.com

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Hardness

Revisit: Feb. 26 (Mechanical Properties)

Measures the resistance to plastic deformation

– NOT toughness!!! NOT strength!!! Measured with small indentations



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Source: Wikimedia Commons: Vickers hardness test schematic, indenter, and indentation on steel

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Welding and the HAZ



Courtesy of Stan T. Mandziej. Used with permission.

- HAZ: Heat Affected Zone
- Gets heat treated by virtue of proximity to weld
- Much cracking happens here!

Source: http://www.china-weldnet.com/English/information/II-1553-05.htm

Processing Methods

Images removed due to copyright restrictions.

Source: http://sifco100.com/when-only-a-hammer-will-do/

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Casting



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- Cooling molten metal from a melt
- Huge grains,
 dendrites from
 cooling
- <u>Left</u>: A meteorite, the ultimate cast microstructure

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Rolling



Courtesy of Elsevier, Inc., http://www.sciencedirect.com. Used with permission. Source: Zheng, C et al. "Microstructure Prediction of the Austenite Recrystallization during Multi-pass Steel Strip Hot Rolling: A Cellular Automaton Modeling." *Computational Materials Science* 44 (2008): 507-14.

Source: C. Zheng et al. Comp. Mat. Sci. 44(2):507 (2008).

- Directionalized force
 - Can elongate grains,
 induce
 recrystallization
 (reformation of grains)
- Can be done hot or cold

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Forging

Image of forging removed due to copyright restrictions.

- Breaking up the microstructure with mechanical force
- Usually done hot
- Reduces grain size
- Directionally works steel

Source: themidnightcarver.blogspot.com/2011/01/forged.html

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Forging

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Source: http://sifco100.com/when-only-a-hammer-will-do/

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How Do Things Fail?

Revisit: Mar. 3 (Failure)

- 1. Overload
- 2. Creep Rupture
- 3. Fatigue

Everyone

- 4. Brittle Fracture
- 5. Wastage
- 6. Environmentally Enhanced
 - General Corrosion
 - Stress Corrosion Cracking
 - Hydrogen Embrittlement
 - Corrosion Fatigue
 - Intergranular Attack
 - Erosion/Corrosion
 - Creep/Fatigue Interaction
 - Liquid Metal Embrittlement
 - Pitting
 - Fretting
 - Dezincification

7. Radiation Damage

-Swelling

- Just us...
- -Void nucleation -Helium embrittlement
- -Radiation induced segragation
- Irradiation creen
- -Irradiation creep
- -Loss of ductility
- -Defect accumulation
- -Fracture toughness reduction
- -Activation
- -Phase dissolution

Creep: Background 0-D and 1-D Defects



Revisit: Feb. 24 (Defects, Dislocations)



Edge Dislocation

Courtesy of Leonid V. Zhigilei. Used with permission.

Source: www.people.virginia.edu/~lz2n/ mse201/mse201-defects.pdf

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Creep



Source: Wikimedia Commons

Revisit: Mar. 3 (Creep, Plasticity)

- Plastic flow under constant stress
- Tension, gravity...
- Happens well below yield stress
- Multiple modes (Coble, Nabarro-Herring...)

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Fatigue

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Repeated application of stress

- Can cause cracks to grow
- Induced by
 vibrations,
 mechanical loading
 Telltale "fatigue striations"

Source: www.sv.vt.edu/classes/MSE2094_NoteBook/97ClassProj/anal/kelly/fatigue.html

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Fatigue: S-N Curves

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Stress (S) vs.
 number of cycles
 (N)

 Lower limit of stress (where N is infinite) is the "safe zone"

Source: www.nde-ed.org/EducationResources/CommunityCollege/Materials/Mechanical/S-NFatigue.htm

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Ductile/Brittle Fracture

Revisit: Mar. 3 (Fracture)



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Ductile Fracture

www.ijme.us/issues/spring2004/IJME_Biomaterial_Revised2.htm

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Brittle Fracture http://site.christensenmaterials.com/Services.html

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Types of Corrosion

See 22.72J (Corrosion)



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Flow Assisted Corrosion

See 22.72J (Corrosion)



Courtesy of Elsevier, Inc., http://www.sciencedirect.com. Used with permission. Source: Balbuad-Celerier, F., and F. Barbier. "Investigation of Models to Predict the Corrosion of Steels in Flowing Liquid Lead Alloys." *Journal of Nuclear Materials* 289, no. 3 (2001): 227-42. Corrosion can be limited by solubility in solvent

- Flow can transport mass away, letting more dissolve
- Fast flow can erode your materials

Source: F. Balbaud-Celerier and F. Barbier. J. Nucl. Mater., 289(3):227-242 (2001).

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SCC

See 22.72J (Corrosion)



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Source: P. Combrade, INEST SCC Workshop, Idaho Falls, ID (2013).

Stress corrosion cracking Requires tensile stress, susceptible

material, aggressive environment

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Proposed SCC Mechanisms

See 22.72J (Corrosion)



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SG Tube SCC Crack

See 22.72J (Corrosion)



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Revisit: Mar. 5 (Radiation Damage)



Adapted, in part, from T. Allen in: "Materials Reliability and Degradation Management Issues In Nuclear Power Plants, May 4-7, Materials Aging Institute, France

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In Nuclear Power Plants, May 4-7, Materials Aging Institute, France

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Radiation Damage: DPA



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Adapted, in part, from T. Allen in: "Materials Reliability and Degradation Management Issues In Nuclear Power Plants, May 4-7, Materials Aging Institute, France

Revisit: Mar. 10 (Stopping Power, DPA)

- DPA Displacements per atom
- Measures average
 number of times
 each atom is
 knocked out of
 position

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Radiation Damage: DPA vs. Properties

LASREF, 40 C

RTNS-II, 90 C OWR, 90 C **Revisit: Mar. 12 (Radiation Effects)**

- Mechanical
 properties change as
 a function of DPA,
 temperature
 - Normally reduction
 in strength, higher
 Young's modulus,
 lower fracture
 toughness



Adapted, in part, from T. Allen in: "Materials Reliability and Degradation Management Issues In Nuclear Power Plants, May 4-7, Materials Aging Institute, France

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300

250

200

150

100

50

0

YIELD STRESS CHANGE, MPa

Radiation Damage: DPA vs. Properties

Revisit: Mar. 12 (Radiation Effects)



Adapted, in part, from T. Allen in: "Materials Reliability and Degradation Management Issues In Nuclear Power Plants, May 4-7, Materials Aging Institute, France Mechanical properties change as a function of DPA, temperature

Normally reduction
in strength, higher
Young's modulus,
lower fracture
toughness

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Radiation Damage: Defect Accumulation

Revisit: Mar. 12 (Radiation Effects)



Void creation



100 nm

Carbide precipitation

Radiation Damage: Void Swelling

Revisit: Mar. 12 (Radiation Effects)

- Void creation (defect accumulation)
 combined with
 hydrogen/helium
 stabilization
- Causes actual dimensional changes!



FLUENCES

BEYOND FFTF GOAL

Adapted, in part, from T. Allen in: "Materials Reliability and Degradation Management Issues In Nuclear Power Plants, May 4-7, Materials Aging Institute, France

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20% CW 316

UNIRRADIATED

CONTROL

Radiation Damage

Revisit: Mar. 12 (Radiation Effects)



Courtesy of Elsevier, Inc., http://www.sciencedirect.com. Used with permission. Source: Was, G. S., et al. "Assessment of Radiation-induced Segregation Mechanisms in Austenitic and Ferritic-martensitic Alloys." *Journal of Nuclear Materials* 411, no. 1-3 (2011): With temperature 41-50.

Radiation-induced segregation in 300-series stainless steel Source: G. S. Was et al., J. Nucl. Mater., 411(1–3), pp. 41-50, 2011.

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